

# Cross Section Spatial Discretization for Nuclear Engineering Calculations

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# Outline

- 1 Equations
- 2 Problem
- 3 Numerical Schemes
- 4 Results
- 5 The End

# Radiation Transport Equation

Ideally we could easily solve the (7-dimensional) Boltzmann neutron transport equation (and its variants):

$$\frac{1}{v} \frac{\partial \psi}{\partial t} + \vec{\Omega} \cdot \nabla \psi + \sigma_t \psi = \int_0^\infty \int_{4\pi} \sigma_s(\vec{\Omega}' \rightarrow \vec{\Omega}, E' \rightarrow E) \psi d\vec{\Omega}' dE' + Q, \quad (1)$$

to answer all of the following questions:

- Where is heat being produced in the nuclear reactor?
- Am I treating this cancer effectively?
- Where is the oil/gas in this well?
- How am I compressing/heating this deuterium pellet?
- What's hiding in the shipping container?
- ...

# 1-D Steady State $S_N$ Equation

Eq. (1) requires more justice than 20 minutes can give.

- Instead, let's talk about:

$$\mu_d \frac{\partial \psi_d}{\partial x} + \sigma_t(x) \psi_d = \frac{\sigma_s(x)}{2} \phi(x) + Q_d(x), \quad (2)$$

the steady state, mono-energetic, 1-D Cartesian geometry,  $S_N$  approximation to Eq. (1) and how solution techniques for Eq. (2) can be improved.

In Eq. (2):

$$\begin{aligned} \psi_d &= \psi_d(x, \mu_d) \\ Q_d &= Q(x, \mu_d) \\ \phi(x) &\approx 2\pi \sum_{d=1}^N w_d \psi_d(x, \mu_d) \end{aligned}$$

# DFEM Discretization

When we discretize Eq. (2) with discontinuous finite elements (DFEM), we get a system of equations like this:

$$\mu_d \mathbf{L} \vec{\psi}_d + \mathbf{R}_{\sigma_t} \vec{\psi}_d = \frac{1}{2} \mathbf{R}_{\sigma_s} \vec{\phi} + \vec{q}_d + \psi_{in} \vec{f}$$

where we define the following (focusing only on  $\mu_d > 0$ ):

$$\begin{aligned} \psi_d(s) &\approx \tilde{\psi}(s) & \mathbf{L}_{ij} &= B_i(1)B_j(1) - \int_{-1}^1 \frac{dB_i}{ds} B_j(s) ds \\ \tilde{\psi}(s) &= \sum_{j=1}^{P+1} \psi_j B_j(s) & \mathbf{R}_{\sigma,ij} &= \frac{\Delta x}{2} \int_{-1}^1 \sigma(s) B_i(s) B_j(s) ds \\ B_j(s) &= \prod_{\substack{k=1 \\ k \neq j}}^{N_P} \frac{s - s_k}{s_j - s_k} & \vec{f}_i &= B_i(-1) \\ & & \vec{q}_{d,i} &= \frac{\Delta x}{2} \int_{-1}^1 B_i(s) Q_d(s) ds \end{aligned}$$

# Cell-wise Constant Cross Sections

If cross section is truly cell-wise constant, there is no approximation in assuming

$$\mathbf{R}_\sigma = \sigma \mathbf{M}$$

- Good examples: shielding calculations, material detection problems

**There are many problems of interest where this is not the case**

- Cross sections are functions of temperature, density, fuel burn-up, etc.
- Neutronics examples: coupled reactor physics, fuel depletion problems, ...
- Radiative transfer: ICF, astrophysics, ...

# Radiative Transfer with Constant Opacities

Initial dissertation topic

- $S_N$  coupled to Euler equations

First step

- Test radiative transfer in MATLAB

Expected result

- Replicate published calculation

Actual result

- Radiation profile within visual norm acceptance criteria
- Temperature profile is not
- Why? WHY?

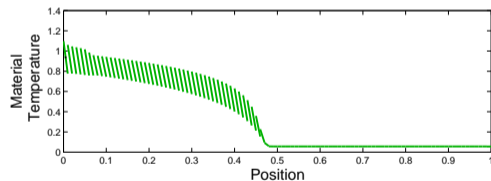
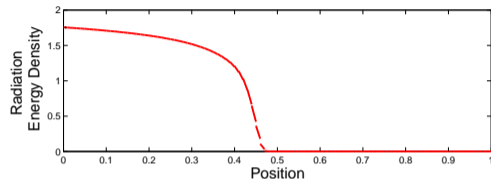


Figure: Cell-wise constant opacity solution to Marshak wave problem with  $\sigma_a = \sigma_t = T^{-3}$ .

# Relationship Between Neutronics and Radiative Transfer

- The 1-D grey radiative transfer equations are:

$$\frac{1}{c} \frac{\partial I}{\partial t} + \mu \frac{\partial I}{\partial x} + \sigma_t I = \frac{\sigma_s}{4\pi} \phi + \frac{\sigma_a}{4\pi} acT^4$$

$$C_v \frac{\partial T}{\partial t} = \sigma_a (\phi - acT^4)$$

- Temperature equation is driven by an interaction term,  $\sigma_a \phi$
- Radiative transfer is more complicated and computationally intense than neutronics.

## Idea

Find a neutronics problem that exhibits behavior similar to radiative transfer temperature profile and demonstrate that the temperature profile is not a result of my own mistake.



# Numerical Schemes to Compare

- **CXS DFEM**: Equally-spaced points as DFEM interpolation points, exact integration, evaluate  $\mathbf{R}$  using cell-wise constant (volume average) cross section
- **SL Gauss**: Gauss quadrature as DFEM interpolation points, evaluate  $\mathbf{R}$  using self-lumping quadrature
- **SL Lobatto**: Lobatto quadrature as DFEM interpolation points, evaluate  $\mathbf{R}$  using self-lumping quadrature
- **SL Newton-Cotes**: Equally-spaced points as DFEM interpolation points, evaluate  $\mathbf{R}$  using self-lumping quadrature

# What is Lumping?

- Matrix lumping, or just lumping, is a mathematical technique that can be applied to all equations solved with DGFEM (and interpolatory basis functions)
- Done to improve the “robustness” of numerical schemes
- In radiation transport, we define robustness as solution positivity and resistance to oscillations
- Lumping- make diagonal matrices
- Two ways to lump
  - ① Collapse an exactly integrated matrix's entries to the main diagonal
  - ② Use quadrature restricted to the DFEM interpolation points

# Self-Lumping Concept

With Lagrange (interpolatory) basis functions, restricting quadrature to the DFEM interpolation points creates a diagonal mass matrix *automatically*

## Self-lumping (SL) $\mathbf{M}$

$$\mathbf{M}_{ij} = \begin{cases} \frac{\Delta x}{2} w_i & i = j \\ 0 & \text{otherwise} \end{cases}$$

Trivial to extend quadrature integration to include spatial variation of cross section

## Self-lumping (SL) $\mathbf{R}_\sigma$

$$\mathbf{R}_{\sigma,ij} = \begin{cases} \frac{\Delta x}{2} \sigma(s_i) w_i & i = j \\ 0 & \text{otherwise} \end{cases}$$

# Test Problem

- Source-free pure absorber with:

$$\sigma_t(x) = c_1 e^{c_2 x}$$

- Vacuum BC on the right and beam of incident flux,  $\psi_{in,d}$ , on the left side in the direction  $\mu_d$ . Analytic  $\psi$  is:

$$\psi(\mu_d, x) = \psi_{in,d} \exp \left[ \frac{c_1}{\mu_d c_2} (1 - e^{c_2 x}) \right]$$

- Interaction rate,  $IR(x)$ , driven only by beam:

$$IR(x) = \sigma_t(x) \psi(\mu_d, x)$$

- In results that follow, we consider  $\mu_d = 1$  and  $x \in [0, 1 [cm]]$

# State of the Practice Solution

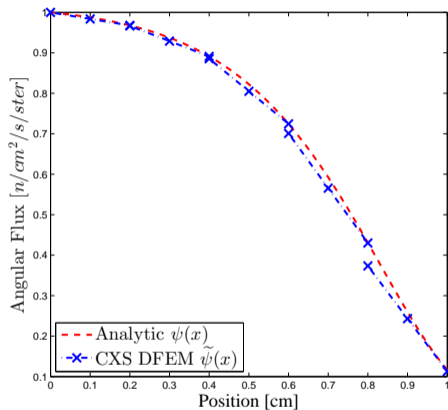


Figure: Angular flux profile.

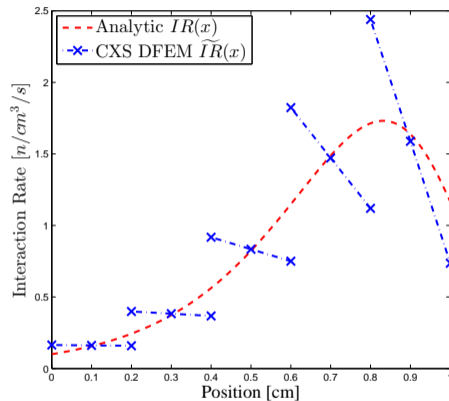


Figure: Interaction rate profile.

**Neutronics problem that exhibits blading seen in radiative transfer!!**

# Surely this is well documented, right?

No. Angular flux profile is smooth. Simplified plotting can hide the blading.

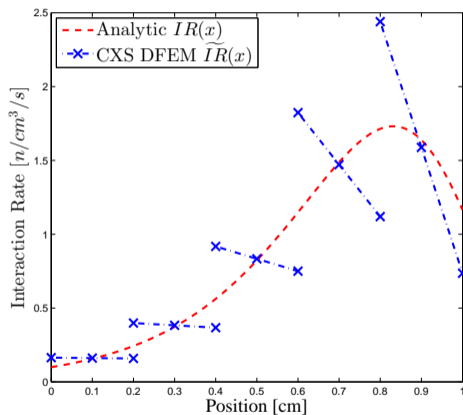


Figure:  $\widetilde{IR}(x)$ .

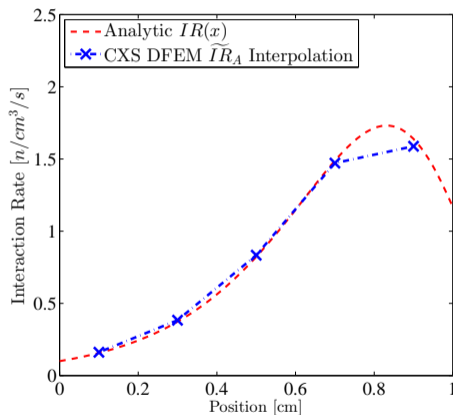


Figure: Interpolated  $\widetilde{IR}$ .

# Something Wrong with DFEM?

No. Consider the analytic solution that uses the cell-wise average cross section.

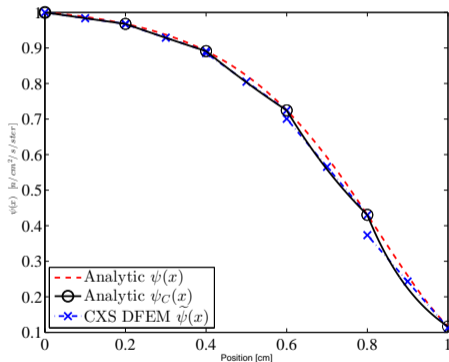


Figure: Angular Flux.

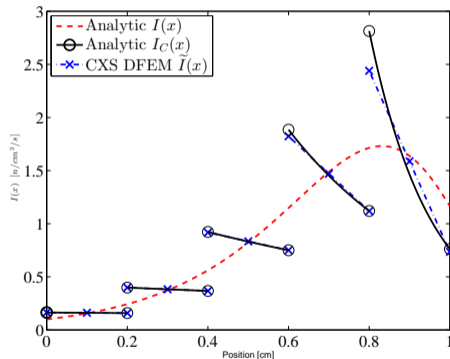


Figure: Interaction Rate.

# Is there a solution to this problem?

Yes. Self-lumping schemes do not exhibit blading. Consider linear SL Lobatto solution:

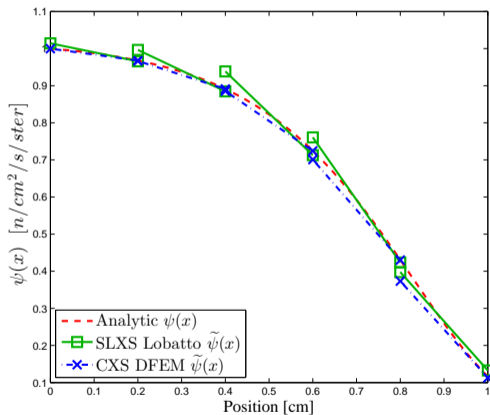


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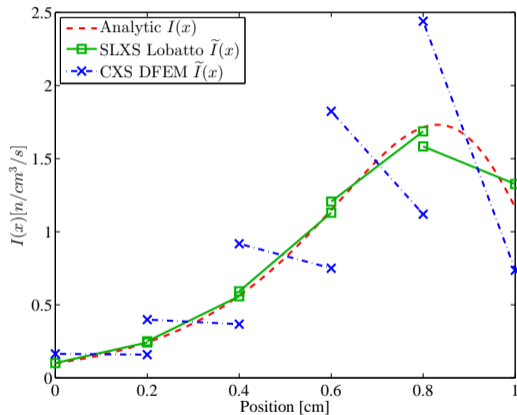
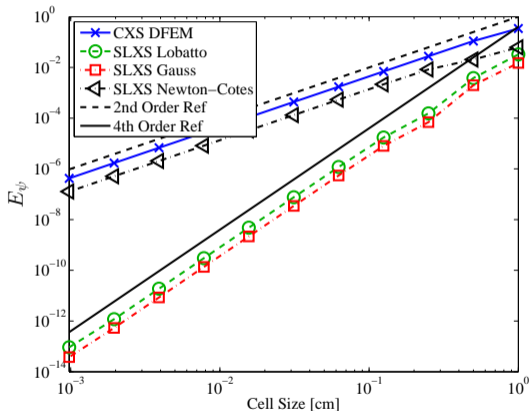


Figure: Interaction Rate.



# $\|\psi - \tilde{\psi}\|_{L^2}$ Convergence

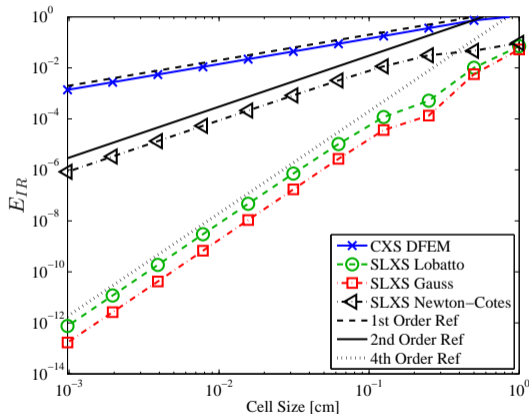


## Summary of Convergence Orders

- SL Gauss:  $\propto P + 1$
- SL Lobatto:  $\propto P + 1$ , less accurate than SL Gauss
- SL Newton-Cotes:  $\propto 2$  if odd  $P$ ,  $\propto 3$  if even  $P$
- CXS DFEM:  $\propto 2$  regardless of  $P$

Figure: Cubic DFEM error convergence for  $\psi(x)$ .

# $\|IR - \tilde{IR}\|_{L^2}$ Convergence



## Summary of Convergence Orders

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Figure: Cubic DFEM error convergence for  $IR(x)$ .

# Marshak Wave Problem with Self-Lumping

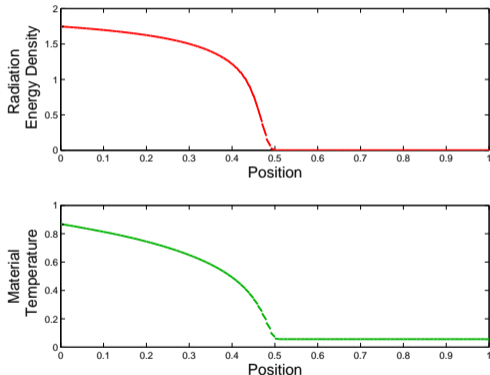


Figure: Linear SL Lobatto solution.

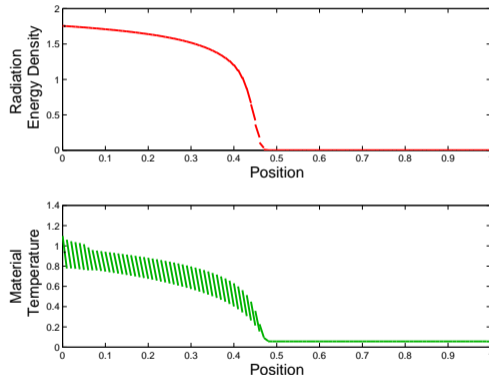


Figure: Linear, constant cross section (with mass matrix lumping) solution.

# Thanks!

Thanks to:

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- Krell Institute and the DOE CSGF

